RAD Conference Proceedings, vol. 3, pp. 21–24, 2018 ISSN 2466-4626 (online) | DOI: 10.21175/RadProc.2018.05

www.rad-proceedings.org



DETERMINATION OF ²²⁶Ra AND ²²⁸Ra IN WATER USING LabSOCS EFFICIENCY CALIBRATION SOFTWARE FOR GAMMA SPECTROMETRIC ANALYSIS

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Abstract. Radium (Ra) isotopes are important from the viewpoints of radiation protection and environmental protection. Radium is routinely analyzed in drinking water. Radium isotopes can be analyzed by different analytical methods based on gamma spectrometric measurements or alpha spectrometry. Modern gamma spectrometry systems are typically operated via computer software applications. In this study, the LabSOCS (Laboratory Sourceless Calibration Software) mathematical efficiency calibration software is used. An improved method was developed to determine radium isotopes from water using gamma spectrometry after micro-coprecipitation as lead (radium) sulfate method for radiochemical separation (Pb(Ra)(Ba)SO₄). This method was successfully modified to allow direct determination of ²²⁶Ra and ²²⁸Ra by gamma spectrometry. However, large volumes of samples and/or long waiting time (20 days), before radioactive equilibrium is established, are required for accurate ²²⁶Ra activity concentration determination. The amounts of ²²⁶Ra and ²²⁸Ra on the sample were quantified by using gamma spectrometric analysis for its 186 keV gamma emission, 351.9 (²¹⁴Pb), 295.2 (²¹⁴Pb), and 609.3 (²¹⁴Bi), 1764.5 (²¹⁴Bi) and for ²²⁸Ra can be nonetheless achieved via its daughter nuclide ²²⁸Ra in 911.2 and 969 keV gamma emission. The radiochemical recovery was 93% and 100% for ²²⁶Ra and ²²⁸Ra, respectively. The Minimum Detectable Activities (MDAs) for 8L of sample and a measuring time of 2.3 days for ²²⁶Ra and ²²⁸Ra were calculated. The best results were obtained when the herein described method was combined with LabSOCS calculations.

 $\textbf{\textit{Key words:}} \ \textit{Radium in water, gamma-ray spectrometry, radiochemical procedure, LabSOCS}$

1. Introduction

The ²²⁶Ra and ²²⁸Ra are the most likely isotopes to be found in drinking water and they are more commonly found in drinking water supplies derived from groundwater where chloride, carbonate and sulphate anions (among others) tend to increase the mobility of radium [1]. In this study, the LabSOCS (Laboratory Sourceless Calibration mathematical efficiency calibration software is used. Methods for the determination of radium isotopes in environmental solids by gamma spectrometry can be obtained by two means, the direct measurement of the principal photopeaks (186 keV in the case of 226Ra), and indirect measurement using the gamma radiation emitting progeny [2]. The advantage of using the direct measurement method, particularly for ²²⁶Ra, is that the sample can be prepared and measured immediately without the need for progeny ingrowth.

The disadvantages of this method relate primarily in that the emission probability of this gamma photopeak is quite weak (3.6%), and furthermore by the presence of the interfering primary gamma emission of ²³⁵U at 185.7 keV that has a higher photon emission probability of 57%. The energy resolution of currently available HPGe spectrometers is generally

considered insufficient to separate the two peaks. If no chemical separation is carried out initially, quantification of $^{226}\mathrm{Ra}$ via the 186.2 keV gamma line must be conducted by attempting to quantify the contribution of the $^{235}\mathrm{U}$ isotope to the doublet peak. In our study, micro-coprecipitation as lead (radium) sulfate method for chemical separation is used.

After the coprecipitation step, the radium isotopes are measured by gamma spectrometry. ²²⁶Ra can be determined by gamma spectrometry by exploiting its gamma emission at 186.2 keV. Lower detection limits are achieved by analyzing its daughter nuclides – lead-214 (²¹⁴Pb) and bismuth-214 (²¹⁴Bi) – with gamma lines with higher emission probabilities. By sealing a radium source, radon-220 (²²⁰Rn) can be contained, and, after 20 days, the radioactive equilibrium of ²²⁶Ra with its daughters ²¹⁴Pb and ²¹⁴Bi is established.

The direct determination of ²²⁸Ra by gamma spectrometry is complicated, since its most important gamma line at 26.4 keV has a very low emission probability of only 1.6 %. An accurate determination of ²²⁸Ra can be nonetheless achieved via its daughter nuclide ²²⁸Ac. This radionuclide has a gamma line at 911.2 keV with a high emission probability of 26.2 %. The radium source is stored for 30 hours before counting in order to allow for the radioactive equilibrium between ²²⁸Ra and ²²⁸Ac to be established.

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2. EXPERIMENTAL

2.1. Radiochemical Procedures

The water sample (minimum 7 kg) was weighed into two 5 L glass beakers. A magnetic stirrer was added and the beaker was put on a hotplate stirrer. 0.3 mL of 133Ba tracer was added into each beaker containing the sample for recovery determination. Concentrated H₂SO₄ (7 mL per L of sample) was added. Pb(Ra)(Ba)SO4 was precipitated by adding 6 mL of the Pb2+ solution (50 mg/mL) dropwise while stirring and heating the solution at 90°C. The white precipitate appeared in a few minutes. If the precipitate did not appear after 20 minutes of stirring, another 1 mL of Pb2+ solution and 5 mL of conc. H₂SO₄ per L of sample were added. The solution was stirred for 3 hours. The stirring was stopped, the stirrer bar was removed and the beaker was covered with watch glass to allow the precipitate to settle down (overnight). The supernatant was decanted to as low a volume as possible. The decanted supernatant was discarded. The precipitate was washed into a 50 mL centrifuge tube with deionized water and then centrifuged at 2500 rpm for 4 minutes. The supernatant was discarded. The precipitate was transferred to a plastic container with 12 mL of deionized water and left under the IR lamp until all of the water evaporated. The plastic container was sealed using liquid glue and left to rest for three days in order to allow for 228Ac ingrowth. The samples were measured for 2.3 days on a coaxial-type gamma spectrometer in order to determine 226Ra via daughters. After 20 days, the samples were remeasured, each for 2.3 days, on the same coaxial-type gamma spectrometer.

2.2. Instrumentation

All samples were prepared in a disc plastic container with the same size of about 3.2 cm in diameter to obtain the similar counting geometry. Gamma spectrometry based on a high-purity germanium detector was used. The HPGe detector has a Full Width at Half Maximum (FWHM) of 1.9 keV for 60Co gamma energy line at 1332 keV and is operated with the Canberra Genie 2000 software for gamma acquisition and analysis. 226Ra was then quantified by gamma counting for its 186 keV gamma emission, 295.2 (214Pb), 351.9 (214Pb), 609.3 (214Bi), 1764.5 (214Bi). For ²²⁸Ra 338.3 gamma emission, 911.2 and 969 keV was used. The gamma spectrometer was calibrated using a standard source with known radionuclides for energy calibration and LabSOCS software for efficiency calibration curve was used. The analysis of the spectra with the known amounts of 226Ra and 228Ra allowed the determination of peak widths, peak resolution and counter efficiencies at different energies and it allowed the determination of counter precision, accuracy and recovery. Two samples are spiked with known amounts of 226Ra and 228Ra and three preliminary samples are spiked with known amounts of 226Ra. All five samples were counted directly in the system with a suitable counting time.

2.3. Description of samples

In the beginning, the proposed radiochemical method was tested with three water samples (DG1, DG2 and DG3) prepared by spiking 4.5 L of tap water with different amounts of a 226Ra standard solution and finally adding a known amount of 133Ba tracer. The final 226Ra activity concentrations in the spiked tap water samples were estimated to vary from 0.14 to 0.37 Bq/L. The objectives of this part were to estimate whether a direct determination of ²²⁶Ra in combination with LabSOCS software is a viable option for routine analysis and to estimate the radiochemical recovery of 133Ba (226Ra and 228Ra). In the second part, a set of 2 water samples, which are denoted as GS1 and GS2 in the tables 2 and 3 that were prepared by spiking 10 L of deionized water with known amounts of 226Ra and ²²⁸Ra standard solutions were analyzed. Contrary to the preliminary test, ²²⁸Ra standard solution was at this stage also added to the samples since it is well known that the gamma emission(s) of its daughter ²²⁸Ac allow an accurate determination of ²²⁸Ra by gamma spectrometry. This part was conducted similarly to an intercomparison run, in the way that the analyst received the samples without knowing their reference activities. For each sample, two aliquots of ca. 7.5 L were taken and analyzed by the radiochemical method described above.

2.4. Determination of Radium concentration

The specific activity (in Bq kg-1), $A_{E\gamma,i}$ of a radionuclide i and for a peak at energy E_{γ} , is given by [3]:

$$A_{E\gamma,\,i} = \frac{NP}{t_c I_\gamma(E_\gamma)\epsilon(E_\gamma)M} \tag{1} \label{eq:alpha}$$

where NP is the number of counts in a given peak area corrected for background peaks of a peak at energy E_{γ} , ϵ (E_{γ}) the detection efficiency at energy E_{γ} , t is the counting lifetime, I_{γ} (E_{γ}) the number of gammas per disintegration of this nuclide for a transition at energy E_{γ} , and M the mass in kg of the measured samples. The procedure is based on microcoprecipitation as lead (radium) sulfate for chemical separation. The yield of the procedure is determined in each sample by adding a ^{133}Ba radioactive tracer. The 356.0 keV γ -ray of ^{133}Ba is used for yield determinations. The highest energy γ -ray from ^{133}Ba is 383.8 keV, which does not interfere with the energies 911 keV from ^{228}Ac .

$$Y_s = \frac{\text{Final mass of barium sulphate}}{\text{Initial mass of barium sulphate}}$$
 (2)

where Y_s is the chemical yield determined for the sample in equation (2).

The calculation of errors is determined by the use of the following equation [4]:

$$\sigma = \frac{\sqrt{N}}{t} = \sqrt{\left(\frac{N}{t^2}\right)} = \sqrt{\frac{r}{t}} \tag{3}$$

where r is the net count rate.

We are interested in subtracting one count from another (gross counts minus background counts) and determining the resulting % error of the NCPS (Net Count Per Second) based on the standard deviation o value, given at 95% confidence level. Thus, Equation (3) is written as:

$$\sigma_n = \sqrt{\frac{r_{oy}}{t_o} + \frac{r_{sy}}{t_s}} \tag{4}$$

where:

 $r_{\rm oy}$, $r_{\rm sY}$ is the net count rate at the gamma line (Y) for background and the sample, $t_{\rm o}$, $t_{\rm s}$ is the real counting time of the background and the sample respectively.

3. RESULTS AND DISCUSSION

3.1. Preliminary test

The preliminary analysis of the measured spectra was conducted by Genie2k software in combination with LabSOCS geometry model (counting efficiency). The radiochemical recovery was calculated by comparing ¹³³Ba photopeak area in the sample spectra with the ¹³³Ba photopeak area in the spectra of a standard source. The obtained results (Table 1) have shown that the values are in good agreement with the expected values independently if the 186 keV photopeak of ²²⁶Ra (samples DG1, DG2 and DG3) or if the photopeaks of its daughters ²¹⁴Pb and ²¹⁴Bi were used for activity concentration calculations (sample DG3 only). The preliminary test confirmed that ²²⁶Ra can be detected and its content accurately determined in a water sample by the herein described method.

Table 1. Results obtained by preliminary tests. Uncertainty is given at k = 2.

Samples	Radio- chemical recovery	Calculated activity [Bq/kg]	Reference activity [Bq/kg]
DG1	1.00 ± 0.05	0.37 ± 0.08	0.37 ± 0.04
DG2	0.96 ± 0.04	0.23 ± 0.06	0.25 ± 0.04
DG3	1.00 ± 0.05	0.16 ± 0.08	0.14 ± 0.02

3.2. Validation of the radiochemical method

In order to further validate the method, two "control" samples were analyzed. The samples were measured 3 days after the source preparation and the measured spectra were at this stage analyzed in two different ways. The LabSOCS geometry modeling was used to simulate detector efficiency curve (results are labeled "LabSOCS" in the tables). The samples are spiked with a known amount of \$^{226}Ra and \$^{228}Ra standard solutions. \$^{133}Ba was used as a radiochemical recovery tracer. The gamma sources were re-measured after 20 days, which allowed for radioactive equilibrium between \$^{226}Ra and its daughters \$^{214}Pb and 214 Bi to be established. The counting efficiency for 214 Pb and 214 Bi was calculated by using LabSOCS software

(results are labeled "LabSOCS – via daughters" in the tables) in order to calculate ²²⁶Ra activity concentrations in the samples. The tables 2 and 3 summarize the ²²⁶Ra and ²²⁸Ra results and their radiochemical recoveries. The results show that ²²⁶Ra activity concentrations determined by the use of the herein described method in combination with LabSOCS software are in agreement with the reference values. The ²²⁸Ra activity concentrations determined by the herein described method in combination with LabSOCS software are all in agreement with the expected values.

Table 2. Results of the 226 Ra analysis. Uncertainty is given at k = 2.

Samples by	Radio-	Calculated	Reference
methods	chemical recovery	activity [Bq/kg]	activity [Bq/kg]
LabSOCS -			
(GS1)	1.00 ± 0.06	0.44 ± 0.08	0.44 ± 0.04
LabSOCS - (GS2)	0.93 ± 0.04	0.75 ± 0.12	0.82 ± 0.06
LabSOCS via daughters - (GS1)	1.00 ± 0.06	0.42 ± 0.06	0.44 ± 0.04
LabSOCS via daughters -			
(GS2)	0.93 ± 0.04	0.73 ± 0.10	0.82 ± 0.06

Table 3. Results of the 228 Ra analysis. Uncertainty is given at k = 2.

Samples by methods	Radio- chemical recovery	Calculated activity [Bq/kg]	Reference activity [Bq/kg]
LabSOCS - (GS1)	1.00 ± 0.06	0.40 ± 0.04	0.37 ± 0.04
LabSOCS - (GS2)	0.93 ± 0.04	0.69 ± 0.04	0.68 ± 0.06

The calculation of Minimum Detectable Activity (MDA) for a given nuclide, at the 95% confidence level, is done according to Currie's formula [5].

$$MDA(Bq/kg) = (2.71 + 4.66(\sigma))/(T*EFF*Y*m)$$
 (5)

where σ is the standard deviation of the background collected during time, T is the collect time (sec), EFF is the efficiency at the energy of interest, Y is Branching Ratio and m is sample mass.

For the herein described method (gamma spectrometric determination) the minimum detectable activities (MDAs) for 8L of sample and a measuring time of 2.3 days are:

- For direct determination of ²²⁶Ra via its 186 kev gamma peak MDAs of 60-70 mBq/kg were calculated.
- For determination of $^{226}\mbox{Ra}$ via its daughters $^{214}\mbox{Pb}$ and $^{214}\mbox{Bi MDAs}$ of 6-8 mBq/kg.
- For determination of 228 Ra via its daughter 228 Ac MDAs of 16-20 mBq/kg.

4. CONCLUSION

This work is aimed to the validation of the methodology for the determination of 226Ra and 228Ra in environmental samples by y-spectrometry. Microcoprecipitation as barium (radium) sulfate was successfully modified to allow for direct determination of 226Ra and 228Ra by gamma spectrometry. However, large volumes of samples and/or long waiting time (20 days), before radioactive equilibrium is established, are required for accurate ²²⁶Ra activity concentration determination. The reasons for the latter are the lower emission probabilities and counting efficiencies when compared to alpha-spectrometric determination. ²²⁸Ra activity concentration determination by the herein described method on the other hand offers some important advantages when compared to, for example, standard radiochemical methods chromatographic separations in combination with liquid scintillation counting technique. The most important advantages are: i) it is not necessary to separate ²²⁸Ra (or its daughter ²²⁸Ac) from other betaemitting radionuclides before counting, thus allowing a simple and fast sample preparation, and ii) due to the fact that secular radioactive equilibrium between ²²⁸Ra and its daughter $^{228}\mathrm{Ac}$ is established in 30 hours, the waiting time between the source preparation and the start of the measurement is relatively short, thus enabling the analyst to obtain results in a reasonable time span. Overall best results were obtained when the herein described method was combined with LabSOCS calculations.

Acknowledgements: The paper is a part of the research done within the project RER/7/008 'Strengthening Capabilities for Radionuclide

Measurement in the Environment and Enhancing QA/QC System for Environmental Radioactivity Monitoring'. The authors would like to thank International Atomic Energy Agency (IAEA) and Labor Spiez for their support.

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 DOI: 10.1021/ac60259a007

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